



# EPRI Low and Intermediate Level Waste Program

Program Overview & Research Update



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Low Level Waste Forum  
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## **Vision**

To be a world leader in advancing science and technology solutions for a clean energy future

## **Mission**

Advancing safe, reliable, affordable, and clean energy for society through global collaboration, science and technology innovation, and applied research.

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# Low and Intermediate Level Waste (LILW) Program Overview

# EPRI Radioactive Waste Management Research

## Generation & Management

Dry Solid Waste

Wet Solid Waste

Mixed waste (hazardous and rad waste)

Liquid radwaste processing and effluents

Radwaste Characterization

## Volume Reduction

Strategies to optimize generation by waste stream/form and classification

Concentration Averaging and Encapsulation

## Storage & Disposal

Disposal site waste acceptance criteria

Very Low Level Waste

On-Site Storage

# Low and Intermediate Level Waste Base Program Overview

## ▪ Recently Completed Projects

- 3002010640, *Guidelines for Operating an Interim On-Site Low Level Radioactive Waste Storage Facility; Revision 2*
- 3002008190, *Comparison of Global Nuclear Power Plant Low and Intermediate Level Radioactive Waste Management and Classification Methods*
- 3002026555, *Accurate Estimation of Technetium-99 and Iodine-129 in Low and Intermediate Level Waste*
- Radwaste Hub
  - <https://radwaste.epri.com>
  - Knowledge transfer and benchmarking site

## ▪ Current Projects / Activities

- Accurate Estimation of Tc-99 and I-129 in Nuclear Power Plant Low and Intermediate Level Waste
- Determining the Capacity of Liquid Radwaste Processing System Resins for Nickel-63
- Impact of Chlorine-36 on Waste Classification and Disposal
- Initial Radioactive Material Shipper Training
- Radwaste Considerations for Pressurized Heavy Water Reactors
- RadBench Database
  - <https://radbench.epri.com>
  - Performance monitoring site

# Low and Intermediate Level Waste Technical Strategy Group

## Activities

- 4 Webcasts/year
- Access to LILW TSG Collaboration site
- Member specific assessment / training
- LILW TSG Research
- Training and Knowledge Transfer

## 2024 Training Sessions

- Radiochemistry for Waste Management
- Waste Characterization
- Liquid Radwaste Fundamentals

## 2024 Research Projects

- Outage Optimization for Radwaste
- Comparison of Waste Characterization Sampling Methods
- Demonstration of Nano Bead Filtration Media
- Irradiated Hardware / Spent Fuel Pool Cleanout Handbook

## 2024 MEMBERS

**Ameren**

**CGN Power Co.**

**CFE (Laguna Verde)**

**Constellation**

**DTE Energy**

**Duke Energy**

**Emirates Nuclear Energy Corporation (ENEC)**

**Entergy**

**Korea Hydro & Nuclear Power Co. (KHNP)**

**Nebraska Public Power District (NPPD)**

**Pinnacle West (Palo Verde)**

**Southern Company**

**STP Nuclear Operating Company**

**Tennessee Valley Authority (TVA)**

**Vistra / Energy Harbor**



**Accurate Estimation of Tc-99 and I-129 in Nuclear  
Power Plant Low and Intermediate Level Waste**

# Background & Purpose

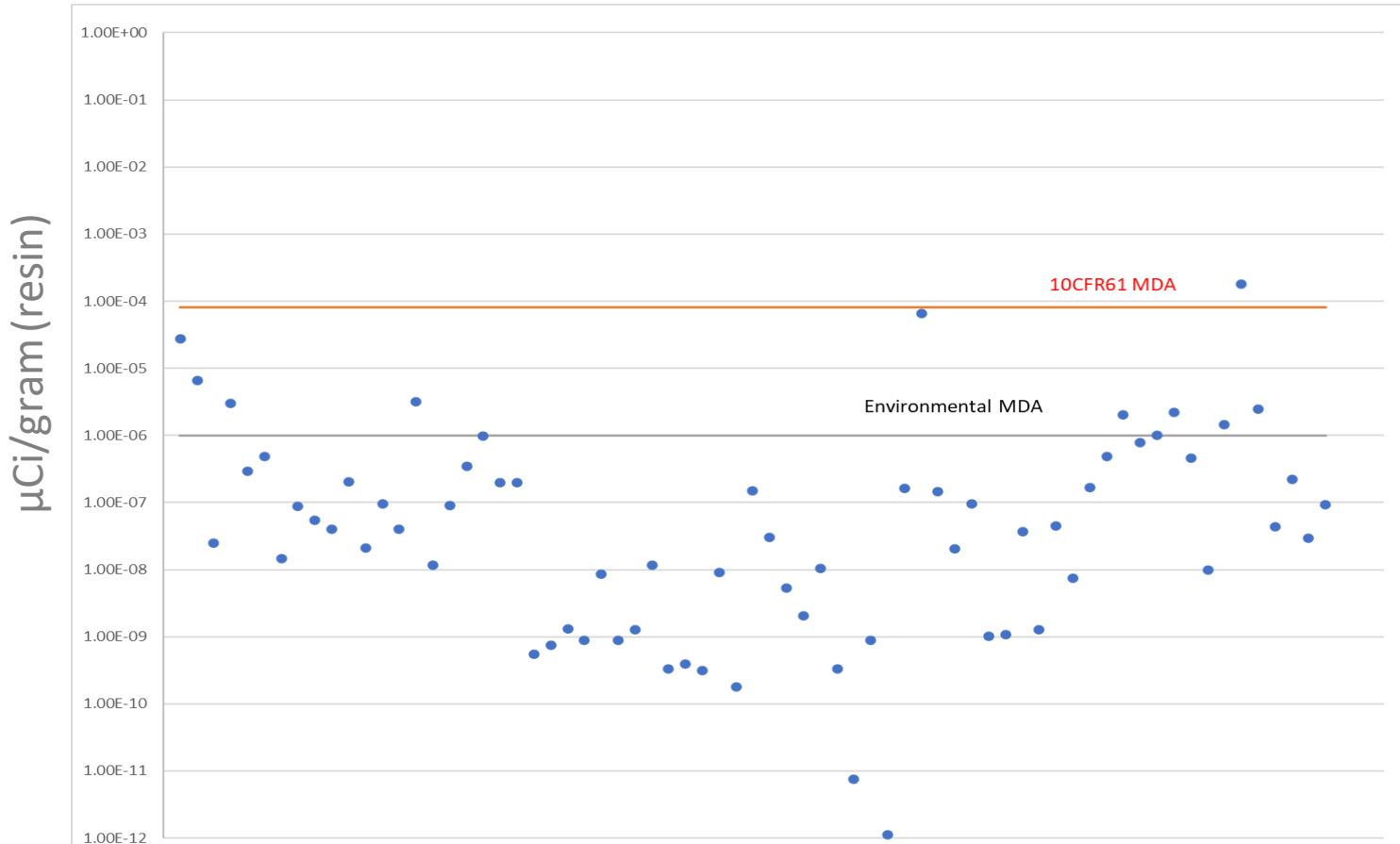
- Tc-99 and I-129
  - Hard-to-detect radionuclides that are required (and important) to characterize for radwaste disposal
- Current Methods of Characterization
  - Lower limit of detection (LLD) of currently commercially available laboratory analysis for Tc-99 and I-129 are several orders of magnitude higher than actual concentrations.
  - Utilities typically use LLD values for waste classification and waste disposal reporting purposes
- Problem
  - Current, typical characterization methods could:
    - inaccurately quantify Tc-99 and I-129
    - limit options for disposal of low and intermediate level waste (LILW) for nuclear power plants

**Objective: Develop more accurate estimation methods for Tc-99 and I-129 in Nuclear Power Plant LILW**



# Current Estimation Practices – U.S. Examples

## NUREG/CR-6567 I-129 Analysis Results Compared to U.S. Commercial Laboratory Minimum Detectable Activity (MDA)



Radionuclide	NUREG/CR-1759 DCF* Value, $\text{Ci}/\text{m}^3$ ( $\text{Bq}/\text{m}^3$ )	ICRP 60 DCF Value, $\text{Ci}/\text{m}^3$ ( $\text{Bq}/\text{m}^3$ )
Tc-99	3.0 ( $1.1\text{E}+11$ )	0.14 ( $5.2\text{E}+09$ )
I-129	0.08 ( $3.0\text{E}+09$ )	0.024 ( $8.9\text{E}+08$ )

\* DCF = Dose Conversion Factor

**EPRI Report 3002003121 – Dose Conversion Factor Evaluation and IMPACTS Analysis of Low Level Radioactive Waste**

# Background & Purpose

- Previous Efforts
  - 1990s: Pacific Northwest National Laboratory (PNNL) measured the Tc-99 and I-129 content in nuclear power plant wastes using mass spectrometric methods (NUREG/CR-6567)
  - 2015: EPRI developed generic scaling factors for Tc-99 and I-129 for LILW using the PNNL dataset (EPRI Report 3002005564)
- 2018 – NRC Request for Additional Information
  - Additional justification needed for:
    - Use of PNNL dataset
    - Statistical methods employed
    - Scaling factor relationships
- 2021
  - NRC expressed openness to review industry submittals of calculated generic scaling factors
  - EPRI emerging issue project funded and prioritized due to impact of potential change in U.S. regulations as well as U.S. and international member feedback

# Phase 1 – Completed

TASK	RESULT
<p>Re-evaluate data from NUREG/CR-6567 to:</p> <ol style="list-style-type: none"> <li>1. Address technical questions related to EPRI Report 3002005564 on generic scaling factors for Tc-99 and I-129.</li> <li>2. Determine an alternative, generic method for more accurately estimating Tc-99 and I-129.</li> </ol>	<ul style="list-style-type: none"> <li>• Sampling and analysis methods are not consistent between samples</li> <li>• Errors identified in NUREG/CR-6567 dataset</li> <li>• EPRI Report 3002005564 removed from epri.com – Generic scaling factor being re-calculated based on new information.</li> </ul>
<p>Explore other datasets (including those from non-US organizations), methods, technologies, and strategies that may be available.</p>	<ul style="list-style-type: none"> <li>• Limited data is available due to inadequate sample representativeness, sampling difficulties, and large expense for analysis</li> <li>• Difference in U.S. versus international disposal site performance assessment</li> </ul>
<p>Publish EPRI Technical Update</p>	<ul style="list-style-type: none"> <li>• Completed – 3002005564</li> </ul>

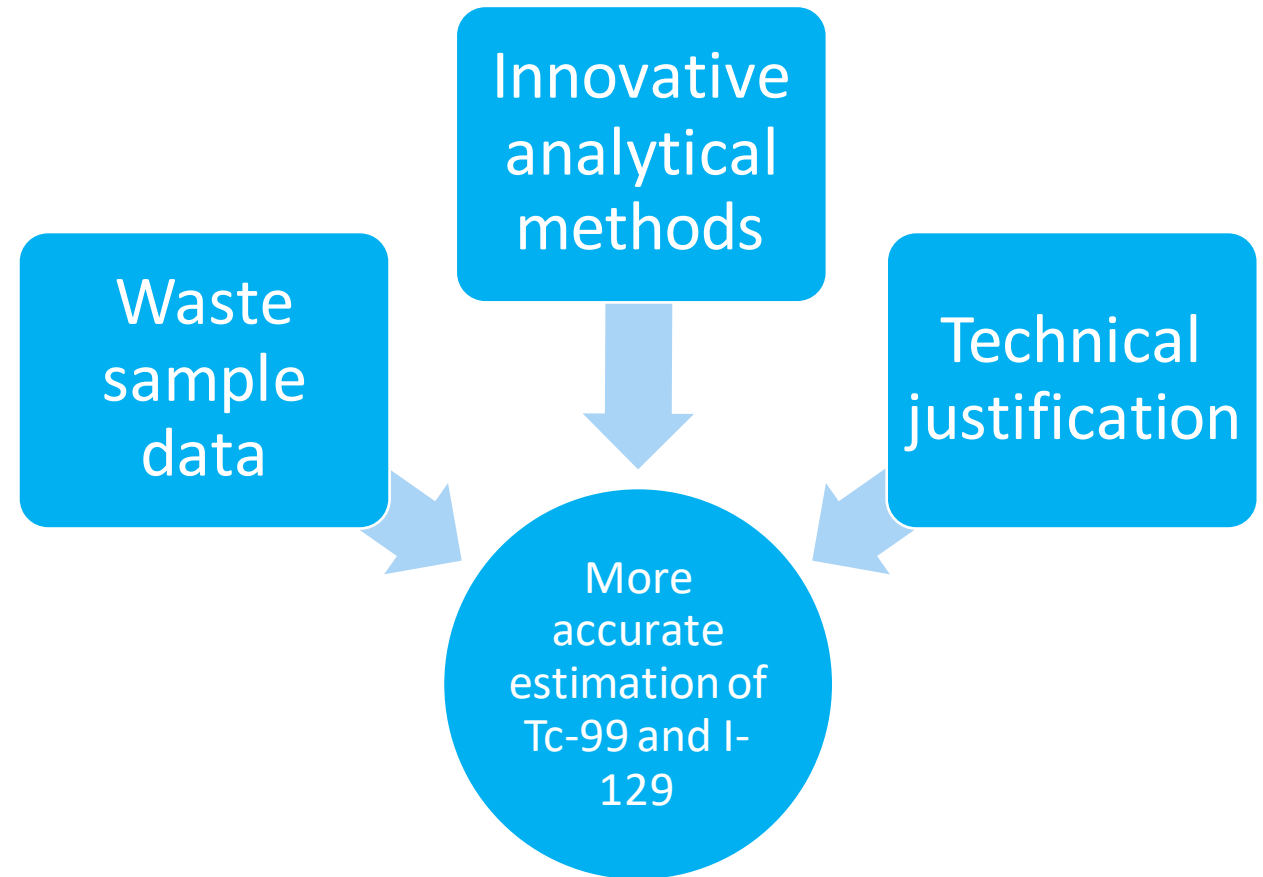
## Phase 2 Initial Task (2024)

- Determine the necessity and/or urgency of more accurate estimation based on recent regulatory developments:
  - Re-calculate the US 10 Code of Federal Regulations (CFR) Part 61, Table 1 and 2 limits using the most recent ICRP Dose Conversion Factors (e.g., ICRP 103)
  - Review non-US disposal facility practices / regulations and related technical justification for the quantification of Tc-99 and I-129.

- Consider the following questions:
  - How do the use of LLDs impact disposal facility capacity? (i.e., are LLDs treated as zeroes?)
  - How do regulatory revisions in progress impact site-specific waste acceptance criteria and waste class limits?
  - What is the impact of using default scaling factor relationships?

# Phase 2 – Path Forward

- Tasks:
  - Collect waste sample data to create a new industry dataset to support statistical analysis of Tc-99 and I-129 in typical plant waste streams
  - Determine if changes can be made to laboratory analytical methods to achieve lower levels of detection for Tc-99 and I-129
  - Review U.S. and international guidance assumptions
  - EPRI Technical Report to capture findings (2025)





# **Impact of Cellulose on Low and Intermediate Level Waste**

# Project Background

- Cellulose:
  - Main constituent of the organic matter that makes up low and intermediate level waste (LILW)
  - Generated in operational and decommissioning nuclear power plants
  - Produces isosaccharinic acid (ISA) which can accelerate nuclide migration
- Application of previous research into regulations is limited and/or unclear
- Knowledge gap exists in U.S. and international waste management industry concerning this issue

# Project Objectives



## Overall Objective:

Identify, collect, and summarize existing research and guidance on the impact of cellulose on LILW



## Supporting Objectives:

Provide information to Radwaste Managers on handling cellulose containing waste and  
Identify where research gaps may exist



# Project Tasks

Role of cellulose (and ISA) on nuclide migration in waste disposal facilities.

Identification of nuclides that may be more susceptible to the impact of cellulose

Identification of existing limits, restrictions, or regulations

Identification of existing methods for detection and/or quantification of cellulose

Correlation between cellulose degradation and waste type

Correlation between cellulose degradation and disposal facility type

Identification of existing methods or strategies for the mitigation or prevention of cellulose impacts

Document relevant operating experience

Correlation between cellulose degradation and waste container type

Identify where research gaps may exist to inform future EPRI LILW research projects

**All findings to be documented in an EPRI Technical Report (2024)**



# Impact of CI-36 on Waste Classification & Disposal

# Cl-36 in Low and Intermediate Level Waste

## Background:

- Half-life =  $3.01 \times 10^5$  years
- NUREG/CR-6567, *Low-Level Radioactive Waste Classification, Characterization, and Assessment: Waste Streams and Neutron-Activated Metals*, Figure 5-4 notes that after about 550 years, Cl-36 becomes the third highest contributor in resin activity for PWRs
- Cl-36 is not included in US waste class limits (10 CFR Part 61)
- IAEA Safety Standard, *Classification of Radioactive Waste*, does address long lived nuclides and allows considerable flexibility on limits based on site and disposal facility needs.

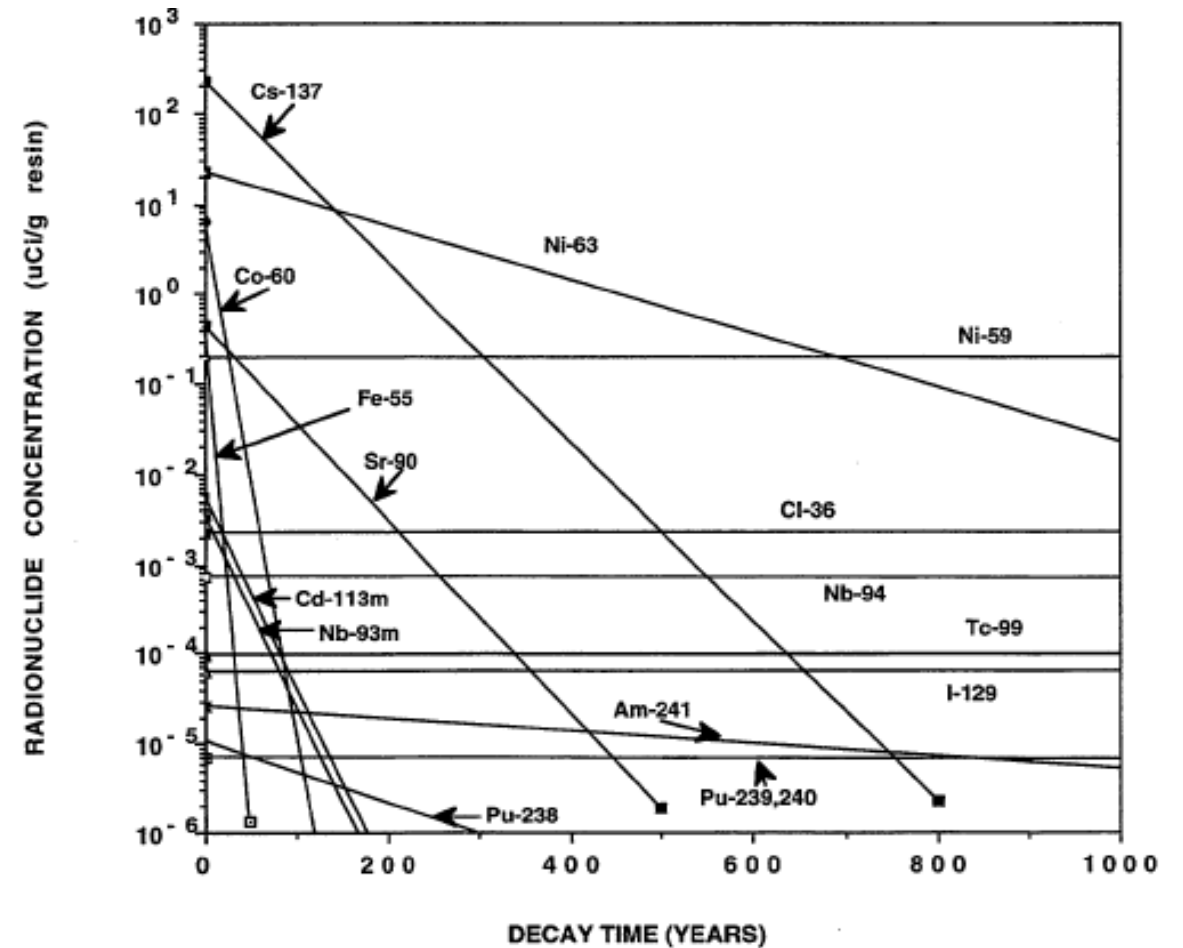


Figure 5.4 Change in Radionuclide Concentration with Time in Crystal River RWCU Resin

Source: NUREG/CR-6567, *Low-Level Radioactive Waste Classification, Characterization, and Assessment: Waste Streams and Neutron-Activated Metals*

# The Why

$^7\text{LiOH}$  supply chain challenges

## Purpose

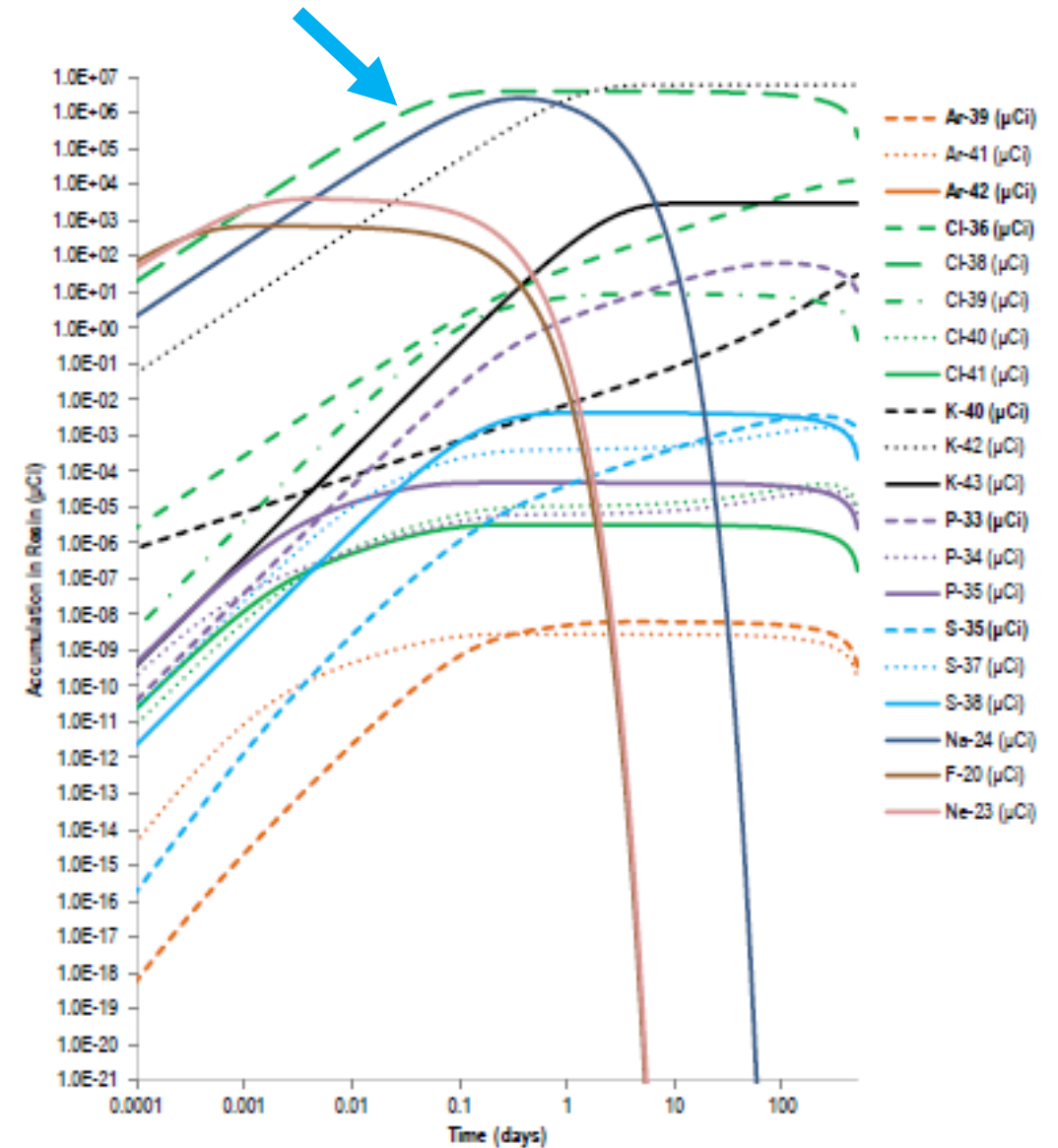
This project is intended to demonstrate the application of KOH for primary system control.

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# Impact of Cl-36 on Waste Classification & Disposal

- Current status: < 300 mCi disposed in the U.S. in the last 20 years
- Cl-36 generation is expected to increase following implementation of KOH



# Project Task Overview

- 2024 – 2025:
  - Collect and analyze waste stream data from the VVER fleet
  - Compile IAEA Member State disposal facility:
    1. Waste acceptance criteria
    2. Disposal volumes of Cl-36
  - Identify key performance assessment modeling parameters for Cl-36
  - Complete modeling for behavior of Cl-36 based on expected generation
- Fall 2025 – Start of KOH application at US PWR Host Site
  - Six months post-application: initial site waste stream review
  - Continue annual waste stream review, dependent upon initial results



# **Initial Radioactive Material Shipper Training Program**

# Initial Radioactive Material Shipper Training Program

- Problem
  - EPRI members continue to be challenged by the time and resources needed to train and qualify personnel to package, ship, and receive radioactive material.
  - Commercially available training courses typically only provide a fundamental introduction to regulatory framework and do not include in-depth learning or practical experience on the necessary steps to complete a radioactive shipment.
- Action
  - Develop a radioactive material shipper training and qualification program that could meet the requirements of:
    - U.S. Department of Transportation (DOT)
    - International Air Transport Association (IATA)
    - International Atomic Energy Agency (IAEA)
    - Institute of Nuclear Power Operations (INPO)
  - Leverage experience from Nondestructive Evaluation (NDE) and Initial Chemistry & RP Training Programs
- Result
  - Elimination of duplication of effort of site qualification programs
  - Improved efficiency and effectiveness of focused training



# Initial Radioactive Material Shipper Training Program

- Approach
  - Utilize the Standardized Task Evaluation (STE) process
  - Industry involvement is KEY. Feedback and agreement is needed on:
    - Task breakdown: what types of shipments / activities would be candidates for this program and highest priority?
    - Support for the ADDIE Process:
      - Analyze
      - Design
      - Development
      - Implementation
      - Evaluation

**2025 – Course Analysis, Design, & Development**

# Initial Radioactive Material Shipper Training Program

- Proposed Pre-requisites:
  - Senior RP Quals or equivalent knowledge (i.e., college degree in applicable field)
  - Leverage EPRI LMS platform and other EPRI Training & Development courses:
    - Math Review
    - Radiation Science Review
- Course Structure
  - Course delivery and length will be determined as part of ADDIE
  - Recorded for future reference and requalification
  - Cost associated with course registration
  - Led by appropriate SMEs (either internal EPRI Staff or contractors)

**2026 – Course Development & Implementation**



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